

Studies and Future Perspectives for Reduction of Radioactive Wastes in the United States

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International Symposium on Present Status and Future Perspective for Reduction of Radioactive Wastes

17 February 2016





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Basis for Research Paths

Recent, Ongoing, and Future Studies

- Aqueous-Based Material Recovery and Waste Reduction
- Electrochemical-Based Material Recovery and Waste Reduction
- Safeguards and Monitoring Technologies
- Future Interests
- Summary





Basis for Research Paths

- U.S. program has the flexibility to study & consider a range of fuel cycle options to understand benefits, limitations, and key issues for research.
- The program is moving forward with select R&D topics to reduce technical risk and uncertainty for future fuel cycle options.





Recent Studies - Aqueous

- The UREX+ family of processes were developed to avoid extraction of pure plutonium.
 - Have the advantage of using well-known chemistry
 - Limitations from lifetime of acetohydroxamic acid (AHA)
 - Five laboratory-scale demonstrations with irradiated fuel completed in the 2003-2007 timeframe
- Development and demonstration of centrifugal contactors have provided enhancements.
 - Contactors provide an alternative to conventional mixer-settlers and pulsed columns
 - High efficiency, short residence time, small space
 - Increased remote maintenance











Ongoing Studies for Aqueous Material Recovery

- The current emphasis is to develop a simplified, robust method for advanced actinide separation.
 - Develop cost effective technology ready for deployment
- The two primary approaches being investigated involve selective complexation of An(III) over the Lns, and the oxidation of Am(VI) to enable a group actinide separation.





Ongoing Studies for Aqueous Material Recovery

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Advanced Waste Forms and Characterization

- R&D for material recovery includes the complete process cycle, including waste forms and behaviors
- Development of fundamental understanding of waste form performance under disposal conditions
- Development of improved processes
- Significant level of international collaboration



Electron Micrograph of Glass Ceramic



Waste package model representation from JAEA



ANSTO Synroc



Glass from Iula Felix, ca 1800 y old



New Studies for Aqueous Material Recovery

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Co-Decontamination Project (CoDCon)

- Demonstrate a separation advanced separations process with irradiated nuclear fuel
- 30/70 plutonium/uranium MOX product
- Study of 4-year duration and beginning in 2016







Recent Studies -Electrochemical

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Electrochemical processing of metallic fuel from the Experimental Breeder Reactor-II (EBR-II)

- Molten salt-based process separates constituents based on electrochemical stability in the chloride phase
 - Low-enriched uranium product
 - Processes at ~10 kg scale





Ongoing Studies for Electrochemical Recycling

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Joint Fuel Cycle Studies

- Assessment of the technical and economic feasibility and nonproliferation acceptability of electrochemical recycling and other options for managing used nuclear fuel
- Used commercial oxide fuel processed to generate metallic uranium and U/TRU products
- Strong R&D as well as demonstration component
- Started in 2011 with 10-year duration







Recent Studies for Electrochemical Recycling

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Development of advanced waste forms and processes tailored for electrochemical recycling processes

- Lead tellurite glasses
- Improved sodalite
- Zinc titanate
- Silica-alumina phosphate



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Safeguards and Monitoring Technologies

- Safeguards and monitoring technologies are important to all separations processes.
- Improvements to methods and technologies are needed.
 - Laser-induced breakdown spectroscopy (LIBS) near real-time composition information (Echem)
 - Triple bubbler real time density, level
 - Voltammetry, actinide sensor for molten salts









Potential Future Interests

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Processing of core debris (Corium)

- Corium can be a complex mixture, but primarily comprised of UO₂, ZrO₂, and siliceous materials.
- Initial studies of corium-type materials indicated that dissolution can be challenging by aqueous processes. Electrochemical techniques may be an alternative.
- Some fission product chemistry may be modestly different in corium versus irradiated fuel, but not expected to be a significant issue.
- Presence of seawater chlorides should not significantly impact electrochemical process.



Simulated Corium Melt

Component		Simulated Corium		Predicted
Oxides	Irradiated LWR Fuel	In-vessel	Ex-Vessel	Electroreduction Partitioning
Major (>10 wt%)				
UO2	X	х	Х	Cathode - metal
ZrO ₂	X	x	х	Cathode – metal
SIO2			X	
Minor (1-5 wt%)				
Cr203		х	X	
CaO			X	Salt
Al ₂ O ₃			X	Cathode – oxide
SrO	x	х	х	Salt
<u>Trace (<<1 wt%)</u>				
Fe ₂ O ₃		x	х	Cathode – metal
MgO			X	Cathode - oxide
CuO		х	х	Cathode – metal
HfO		x	Х	Cathode - metal
K ₂ O			X	Salt
MnO		х	х	Cathode – metal
Na ₂ O			X	Salt
NiO		х	X	Cathode – metal
Ta ₂ O ₅		x	x	Cathode - metal
TiO ₂		х	Х	Cathode - metal
ZnO		х	Х	Cathode - metal



Potential Future Interests

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- Corium treatment via electrochemical processing methods is a potential approach, but requires experimental confirmation.
- Simulated and real materials potentially available for testing.



Corium to metal conversion process development timeline (No separation of actinides from metal product)







- Robust R&D history in material recovery and reduction of radioactive wastes, in both aqueous and electrochemical technologies
- Ongoing studies investigate cutting-edge issues in separations technologies
- Future interest to investigate processing of corium-type materials

