Fuel Safety Research Meeting 2009
Program

Wednesday, May 20

Opening Session		
Opening address	H. Yokomizo	JAEA
Fuel safety research at JAEA	T. Fuketa	JAEA
Session 1 Fuel Behavior under RIA Condition		
Current status of RIA criteria in the United States	J. Voglewede	USNRC
An industry view of RIA criteria in the United States	R. Montgomery	ANATECH
EDF recent works on RIA limits	N. Waeckel	EDF
Lunch break		
Influence of test conditions on rod behaviour during RIA based on CIP0-1, VA-1 and VA-3 tests	V. Georgenthum	IRSN
PCMI failure of high burnup fuels under RIA conditions	T. Sugiyama	JAEA
Numerical analysis on the cladding failure behavior of high burnup PWR fuels in NSRR experiments.	M. Suzuki	JAEA
Coffee break		
Experimental study of the irradiated zircaroy-4 fracture process using RIA dedicated PST specimens and DIC method	S. Carassou	CEA
Synthesis of an international round robin on cladding mechanical testing for RIA	C. Poussard	CEA
RIA failure of high burn-up fuel rod irradiated in KKL	V. Grigoriev	Studsvik
Reception		

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Session 2 Fuel Behavior under LOCA Condition		
A comparison of fuel relocation fragmentation and relocation behaviour in Halden reactor LOCA experiments	E. Kolstad	Halden Project
Recent results at Argonne on embrittlement of high-burnup cladding	Y. Yan	ANL
Current Status of LOCA Cladding Criteria in the United States	J. Voglewede	USNRC
An industry view of LOCA rulemaking in the United States	K. Yueh	EPRI
EDF views on LOCA limits	N. Waeckel	EDF
Lunch break		
IRSN views on LOCA criteria and R&D related programs	M. Petit	IRSN
DRACCAR, a new 3D-thermal mechanical computer code to simulate LOCA transient on Nuclear Power Plants : status of the development and the validation	G. Repetto	IRSN
Fracture resistance and embrittlement of high burnup fuel cladding under LOCA conditions	F. Nagase	JAEA
Session 3 Extended session		
Mechanical properties of hydrogenated cladding tubes	T.Fukuda	JAEA
Coffee break		
Ab initio study on plane defects in Zirconium hydrogen solution and Zirconium hydride	Y. Udagawa	JAEA
Evaluation of uncertainties in FEMAXI-6 calculations with MOX fuels irradiated in Halden reactor	A. Yamaji	JAEA
Fuel irradiation and water chemistry test programs in JMTR	T. Nakamura	JAEA
Closing Session		
Meeting summary	T. Fuketa	JAEA
Closing remarks	K. Ishijima	JAEA