

## 研究成果一覧

### 雑誌掲載論文 :

- 1) T. Kashima, k. Suyama, H. Mochizuki, "Validation of Burnup Calculation Code SWAT4 by Evaluation of Isotopic Composition Data of Mixed Oxide Fuel Irradiated in Pressurized Water Reactor," *Energy Procedia*, 71, 159-167 (2015).
- 2) K. Tonoike, H. Sono, M. Umeda, Y. Yamane, T. Kugo, K. Suyama, "Options of Principles of Fuel Debris Criticality Control in Fukushima Daiichi Reactors," *Nuclear Back-end and Transmutation Technology for Waste Disposal*, Chap. 21, pp. 251-259, Springer, (2015).
- 3) T. Ueki, "Fractal dimension analysis for run length diagnosis of Monte Carlo criticality calculation," *Journal of Nuclear Science and Technology*, 53, p.312 (2016).
- 4) T. Ueki, "Monte Carlo criticality analysis under material distribution uncertainty," *Journal of Nuclear Science and Technology*, 54, p.267 (2017).
- 5) T. Ueki, "A power spectrum approach to tally convergence in Monte Carlo criticality calculation," *Journal of Nuclear Science and Technology*, 54, p.1310 (2017).
- 6) S. Gunji, et al, "Study of experimental core configuration of the modified STACY for measurement of criticality characteristics of fuel debris," *Progress in Nuclear Energy*, 101, p.321 (2017).
- 7) T. Ueki, "Monte Carlo criticality analysis of random media under bounded fluctuation driven by normal noise," *Journal of Nuclear Science and Technology*, 55, p.1180 (2018).
- 8) T. Ueki, "Universal methodology for statistical error and convergence of correlated Monte Carlo tallies," *Nuclear Science and Engineering*, 193:7, p.776-789 (2019).
- 9) T. Ueki, "Judgment on Convergence-in-Distribution of Monte Carlo Tallies Under Autocorrelation," *Nuclear Science and Engineering*, 194:6, p.422-432 (2020).

### 技術報告書 :

- 1) 鹿島陽夫, 須山賢也, 高田友幸, "連続エネルギーモンテカルロコード MVP, MCNP 及び核計算コード SRAC を使用する統合化燃焼計算コードシステム; SWAT4.0," JAEA-Data/Code 2014-028 (2014).

### 国際会議等報告 :

- 1) T. Kashima, K. Suyama, H. Mochizuki, "Validation of Burnup Calculation Code by Evaluation of Isotopic Composition Data of Mixed Oxide Fuel Irradiated in Pressurized Water Reactor," *The Fourth International Symposium on Innovative Nuclear Energy Systems*, November 6-8, 2013, Tokyo, Japan (2013).
- 2) K. Tonoike, et al., "Revival of Criticality Safety Research in Japan Atomic Energy Agency," *Trans. of ANS*, 110(1), p.282 - 285, 2014/06.

- 3) K. Tonoike, "Experience of the JCO Criticality Accident," *IEM9*, April 20-24, 2015 Vienna (2015).
- 4) K. Tonoike and Y. Yamane, "Criticality Control of fuel debris - TMI-2 Review and Fukushima Expectation," *OECD/NEA/CSNI/WGFCS ORACS Workshop*, May 19-21, 2015, Albuquerque, USA (2015).
- 5) K. Tonoike, et al., "Study on Criticality Control of Fuel Debris by Japan Atomic Energy Agency to Support Nuclear Regulation Authority of Japan," *ICNC2015*, Sept. 13-17, 2015, Charlotte, USA (2015).
- 6) Y. Yamane, et al., "Development of Criticality Risk Evaluation Method for Fuel Debris in Fukushima-Daiichi NPS," *ICNC2015*, Sept. 13-17, 2015, Charlotte, USA (2015).
- 7) K. Tonoike, et al., "Criticality Characteristics of MCCI Products Possibly Produced in Reactors of Fukushima Daiichi Nuclear Power Station," *ICNC2015*, Sept. 13-17, 2015, Charlotte, USA (2015).
- 8) Y. Miyoshi, et al., "Present Status of STACY Modification Program and Fundamental Nuclear Properties of Experimental Cores Related to Fuel Debris Criticality," *ICNC2015*, Sept. 13-17, 2015, Charlotte, USA (2015).
- 9) K. Izawa, et al., "Design of Water-Moderated Heterogeneous Cores in New STACY Facility through JAEA/IRSN Collaboration," *ICNC2015*, Sept. 13-17, 2015, Charlotte, USA (2015).
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- 11) S. Gunji, et al., "Study of experimental core configuration of the modigied STACY for reactivity worth measurement of MCCI products," *PHYSOR2016*, May 1-5, 2016, Sun Valley, USA (2016).
- 12) M. Ernoult, et al., "New Model for Parametric Analysis of Fuel Debris Criticality," *PHYSOR2016*, May 1-5, 2016, Sun Valley, USA (2016).
- 13) T. Ueki, "Spectral Analysis for Convergence Assessment in Monte Carlo Criticality Calculation," *M&C2017*, April 16-20, 2017, Jeju Island, Republic of Korea (2017).
- 14) K. Tonoike, et al., "Progress of Criticality Control Study on Fuel Debris by Japan Atomic Energy Agency to Support Secretariat Of Nuclear Regulation Authority," Proceedings of International Conference on Nuclear Criticality Safety (ICNC) 2019 (2019).
- 15) T. Watanabe, et al., "Criticality Characteristics of MCCI Products Possibly Produced in Reactors of Fukushima Daiichi Nuclear Power Station," Proceedings of International Conference on Nuclear Criticality Safety (ICNC) 2019 (2019).
- 16) Y. Nagaya, et al., "SOLOMON: a Monte Carlo Solver for Criticality Safety Analysis," Proceedings of International Conference on Nuclear Criticality Safety (ICNC) 2019 (2019).
- 17) K. Izawa, et al., "Neutronic Design of Basic Cores of the New STACY," Proceedings of International Conference on Nuclear Criticality Safety (ICNC) 2019 (2019).

- 18) S. Gunji, et al., "Design Methodology for Fuel Debris Experiment in the New STACY Facility," Proceedings of International Conference on Nuclear Criticality Safety (ICNC) 2019 (2019).
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- 20) S. Araki, et al., "A New Critical Assembly: STACY," Proceedings of RRFM 2020 European Research Reactor Conference (2020).
- 21) S. Araki, et al., "Effect of  $\beta$  on Effective Multiplication Factor in  $1/f^\beta$  Spectrum Random System," Proceedings of PHYSOR2020 (2020).

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- 1) 佐藤真人, 他, “陰イオン交換分離法を用いた燃焼燃料中の微量希土類元素等の分析手法の予備検討”, 日本原子力学会 2013 年秋の大会(2013).
- 2) 佐藤真人, 他, “燃焼燃料組成測定に資する Sm・Pm 分離手法の検討”, 日本原子力学会 2015 年春の年会(2015).
- 3) 外池幸太郎, 他, “MCCI 生成物の臨界量評価”, 日本原子力学会 2015 年春の年会(2015).
- 4) 植木太郎, “確率的乱雑化による UO<sub>2</sub>・コンクリート系の臨界性評価の揺らぎ”, 日本原子力学会 2016 年春の年会(2016).
- 5) 渡邊友章, 他, “希薄プルトニウム溶液の反応度温度係数及びボイド係数の温度依存性”, 原子力学会 2016 年春の年会(2016).
- 6) 植木太郎, “直交規格化加重に基づく標準化時系列法による統計誤差評価”, 日本原子力学会 2016 年秋の大会(2016).
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- 10) 長家康展, 他, “臨界安全解析用モンテカルロ計算ソルバーSolomon の開発(1) ACE 形式に基づく衝突解析モデルの実装”, 日本原子力学会 2018 年春の年会(2018).
- 11) 植木太郎, “モンテカルロ法臨界計算における統計誤差評価とバイアス補正”, 日本原子力学会 2018 年秋の大会(2018).
- 12) 長家康展, 他, “臨界安全解析用モンテカルロ計算ソルバーSolomon の開発(2) 非分離共鳴断面積に対する確率テーブル法の実装”, 日本原子力学会 2018 年秋の大会(2018).
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- 14) 植木太郎, 他, “確率的乱雑化モデルの拡張と臨界計算コード Solomon への実装”, 日本原子力学会 2019 年春の年会(2019).
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- 16) シマヌルラン・リアプト, 他, “Effect of Gd<sub>2</sub>O<sub>3</sub> on fission number estimation during a severe accident in a BWR spent fuel pool”, 日本原子力学会 2019 年春の年会(2019).
- 17) 植木太郎, “多種物質系乱雑化モデルの開発とモンテカルロソルバー Solomon での検証”, 日本原子力学会 2019 年秋の大会(2019).
- 18) 長家康展, 他, “臨界安全解析用モンテカルロ計算ソルバー Solomon の開発 (3) 熱中性子散乱モデルの実装”, 日本原子力学会 2020 年春の年会(2020).
- 19) トウヤ・デルゲルサイハン, 他, “Adjoint-weighted kinetics parameter calculation using multigroup version of Solomon solver,” 日本原子力学会 2020 年春の年会(2020).
- 20) シマヌルラン・リアプト, 他, “Consequence analysis of a postulated nuclear excursion in BWR spent fuel pool using  $1/f^{\beta}$  spectrum model of randomization,” 日本原子力学会 2020 年春の年会(2020).