

Panel2: U.S.-Japan Cooperation in R&D to Improve Safety of Existing Reactors

R&D activities for LWR-system safety in JAEA

Masahiko Osaka Japan Atomic Energy Agency 6th Symposium on U.S.-Japan Nuclear Energy Research Cooperation

24 February 2023, Washington DC, USA



- 1. Outline of R&D activities for LWR-system safety
- 2. Accident Tolerant Fuel (ATF) R&D program in Japan
 2.1 Overview of ATF R&D in Japan
 2.2 JAEA's role in ATF R&D
 2.3 Irradiation program in US test reactors
 2.4 US-Japan collaboration under CNWG framework
- 3. Summary and future prospect

1. R&D activities for LWR-system safety in JAEA

Utilities, manufacturer, fuel vendor: Nuclear safety enhancement by safety design and management

Utilization promotion

R&Ds for issue-solving

(National Project, Collaborative Researches with Nuclear Industries)



Nuclear Science and

Engineering Center (NSEC), and related organizations in JAEA

Basic and fundamental research:

- Cooperation with university, research institute, industries, etc. -
- Accident Tolerant Fuel (ATF) development, material ageing issue, etc.
- Advanced computational/experimental tools for LWR-system analysis
- Preparation and utilizing unique infrastructure and technologies

<u>Regulators :</u> Promote nuclear safety culture in Operators



R&Ds for issue-solving (NRA Consignment Research)

Nuclear Safety Research Center (NSRC)



*Sponsored by METI

Irradiation of Reactor Pressure Vessel (RPV) samples in JRR-3 for enlargement of irradiation data at high dose region

2022	2023	2024	2025
Preparation	Design and fabrica	tion of	on
	Irradiation capsule	Irradiati	PIE



JRR-3 core



Thermal neutron flux (10¹⁴/cm²/s) https://jopss.jaea.go.jp/pdfdata/JAEA-Review-2017-007.pdf

https://jrr3.jaea.go.jp/6/pdfs/2023_JRR-3.pdf

4

Accident Tolerant Fuel (ATF) R&D program in Japan 2.1 Overview of ATF R&D in Japan

5



JAEA takes charge of project coordination/management of Japanese ATF R&D sponsored by METI



- Project coordination/management of Japanese ATF R&D sponsored by METI
- Development of common technological basis:
 - Irradiation test and Post Irradiation Examination (PIE) technologies
 - Development of fuel behavior analysis code
 - ➢R&D on advanced technologies, supporting fuel behavior analysis under long-term irradiation and accident condition:
 - ✓ Quantum beam (ion, electron, etc.) irradiation combined with in-situ corrosion test
 - ✓ Improvement of out-of-pile LOCA test

2.3 Irradiation program in US test reactors

• Cr-coated Zircaloy: in ATR (CNWG framework, CRADA)

(Japanese fiscal year, JFY)

2021	2022	2023	2024	2025	2026	2027	2028	2029
Parts production, transportation, inspection, etc.								
	Irrad	iation (low	burn-up,	PWR				
	loop-	type, instr	umented)					
Post-Irradiation Examination (PIE) (low burn-up)								
	-	Irradia	tion (high b	urn-up)		PIE (high	burn-up) – – ►	

• FeCrAl-ODS: in ATR (CNWG framework, CRADA)

➢ Preparation of the CRADA contract for the ATR irradiation

- ➢UO₂-fueled pins under loop-type environment, expected irradiation start from 2024 JFY
- SiC/SiC:
 - ➤Contract signed for the MITR irradiation

SiC/SiC samples under loop-type environment, expected Irradiation start from 2023 JFY

2.4 US-Japan collaboration under CNWG framework

[Undergoing]

- Irradiation test at US test reactors:
 ATR: Cr-Zircaloy and FeCrAl-ODS
 - ► MITR: SiC/SiC
- Participation to the annual EPRI/DOE/INL Joint Workshop
 [Prospect]



• Further collaboration under CNWG framework to accelerate the ATF development in both US and Japan, e.g. transient test on ATF, advanced technology development, etc.



- Overview of R&D activities on LWR-system safety in JAEA was introduced.
- We are facing to the important point; The first irradiation test of Japanese ATF in collaboration with US.
 - ➢Cr-coated Zircaloy in ATR, ready for irradiation
 - ➢ FeCrAI-ODS in ATR, under preparation of contract
 - SiC/SiC in MITR, signed on contract, irradiation expected next Japanese fiscal year
- Further collaboration expected under CNWG framework to accelerate the ATF development in both US and Japan



Thank you for your attention



Complementary slides

Outline of ATF development in Japan(1/2)

Background

- In order to enhance the safety of LWR, there is <u>need for developing new fuel with enhanced accident</u> <u>tolerance, e.g. less melting and/or less hydrogenating fuel</u> (Lessons learned from Fukushima Daiichi Nuclear Power Plant Accident)
- ➢ <u>Objective</u>
- <u>Establishment of "Technical Basis" for implementing an accident tolerant fuel (ATF) in the existing</u> <u>LWRs</u>, supporting the industry-lead ATF development.

• <u>Action Item</u>

- Update of ATF technological information such as technological readiness level (TRL), roadmap, international trend, etc.
- Consideration of standard and criteria, followed by ATF-related licensing action
- Development of common technologies (irradiation technology, fuel behavior analysis code etc.) for accelerating ATF implementation

• Research Team

- JAEA : Project coordination, etc.
- Japanese power plant provider, fuel vendor, research institute, university : Collaborators (human resource, know-how, experience, analytical tool, etc.)
- CNWG, Int'l collaboration

- Expected Outcome
- Deployment of ATF → Enhancing LWRs safety (Decreasing potential risk in accident progression)

12

• Technology succession and human resource development in the field of nuclear fuel

Outline of ATF development in Japan(2/2)





- Planning for ATF irradiation in test reactor
 - Negotiation with test reactor operators in US and Europe
 - ✓ Halden & JMTR shutdown....
 - ✓ Seeking opportunity to irradiate Japanese ATF
 - CNWG (Collaboration between US and Japan)
 - ✓ <u>2017-@ORNL</u>

HFIR Irradiation of Japan-developed FeCrAl-ODS Alloys

HFIR Irradiation of Japan-developed FeCrAI-ODS Alloys

